



MSc Energy (2019/20)

NUCLEAR REACTOR TECHNOLOGY (LECTURE 2)

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Neutron Diffusion

- ▶ 1D Diffusion Equation

$$J_z = -D \frac{\partial n}{\partial z}$$

- ▶ 1st Fick's Law

- ▶ 3D Diffusion Equation $\mathbf{J} = \mathbf{i}J_x + \mathbf{j}J_y + \mathbf{k}J_z = -\hat{D}\vec{\nabla}n$

- ▶ Relation of the diffusion coefficient to the scattering length

$$J_z = \frac{v}{6}n(z - \lambda_s) - \frac{v}{6}n(z + \lambda_s) = -\frac{v\lambda_s}{3} \frac{\partial n}{\partial z} = -D \frac{\partial n}{\partial z}$$

$$D = \frac{v\lambda_s}{3}$$

Continuity Equation

▶ Second Fick's Law in 3D:
$$\frac{\partial n}{\partial t} = - \left[\frac{\partial J_x}{\partial x} + \frac{\partial J_y}{\partial y} + \frac{\partial J_z}{\partial z} \right] = - \vec{\nabla} \cdot \mathbf{J}$$

▶ The same in 1D (attention on the D not being constant):

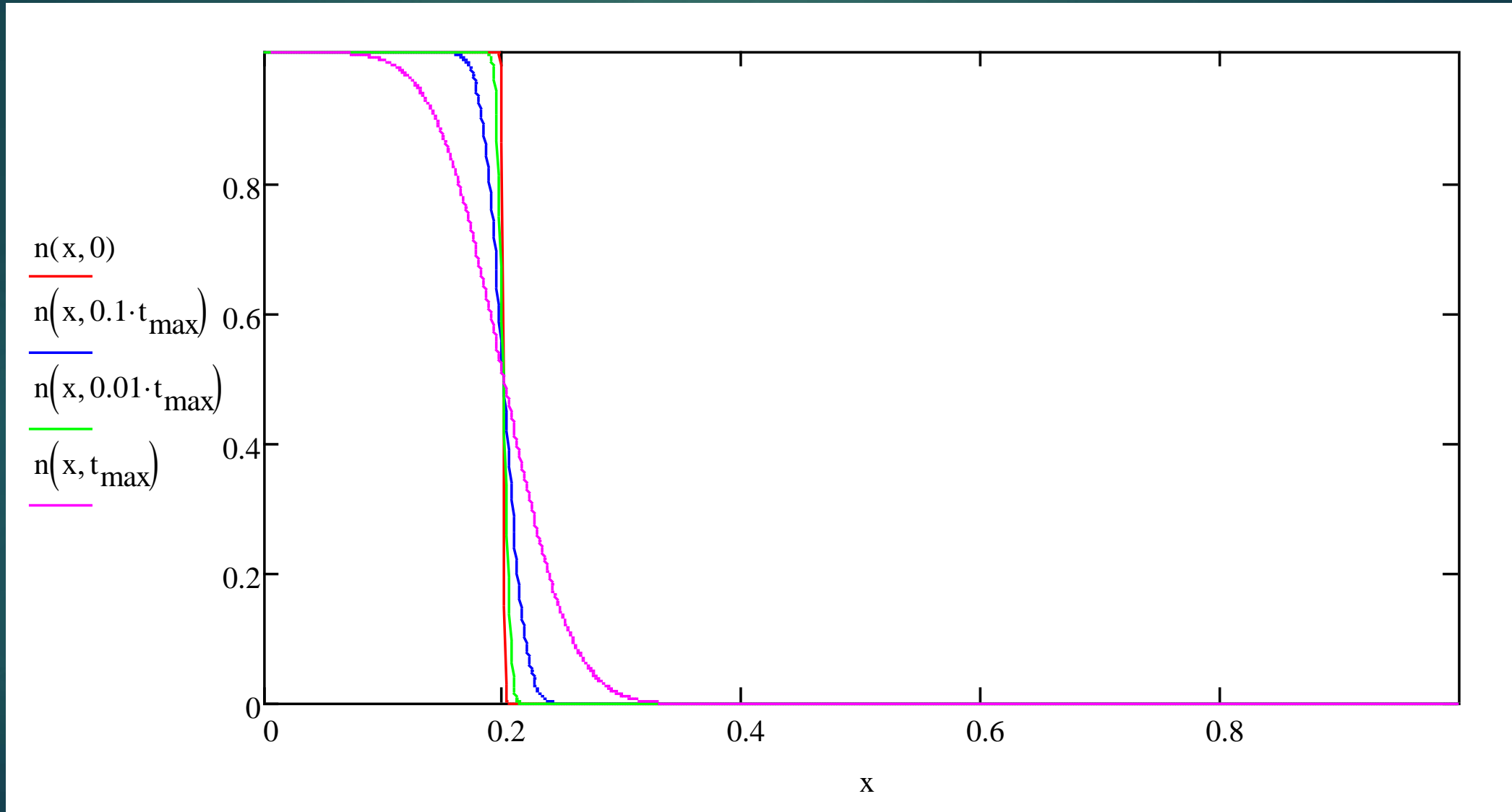
$$\frac{\partial n}{\partial t} = D \nabla^2 n = \Delta D n$$

▶ Time-dependent continuity equation for the neutron flux: $\Phi(\mathbf{r}) = v n(\mathbf{r})$

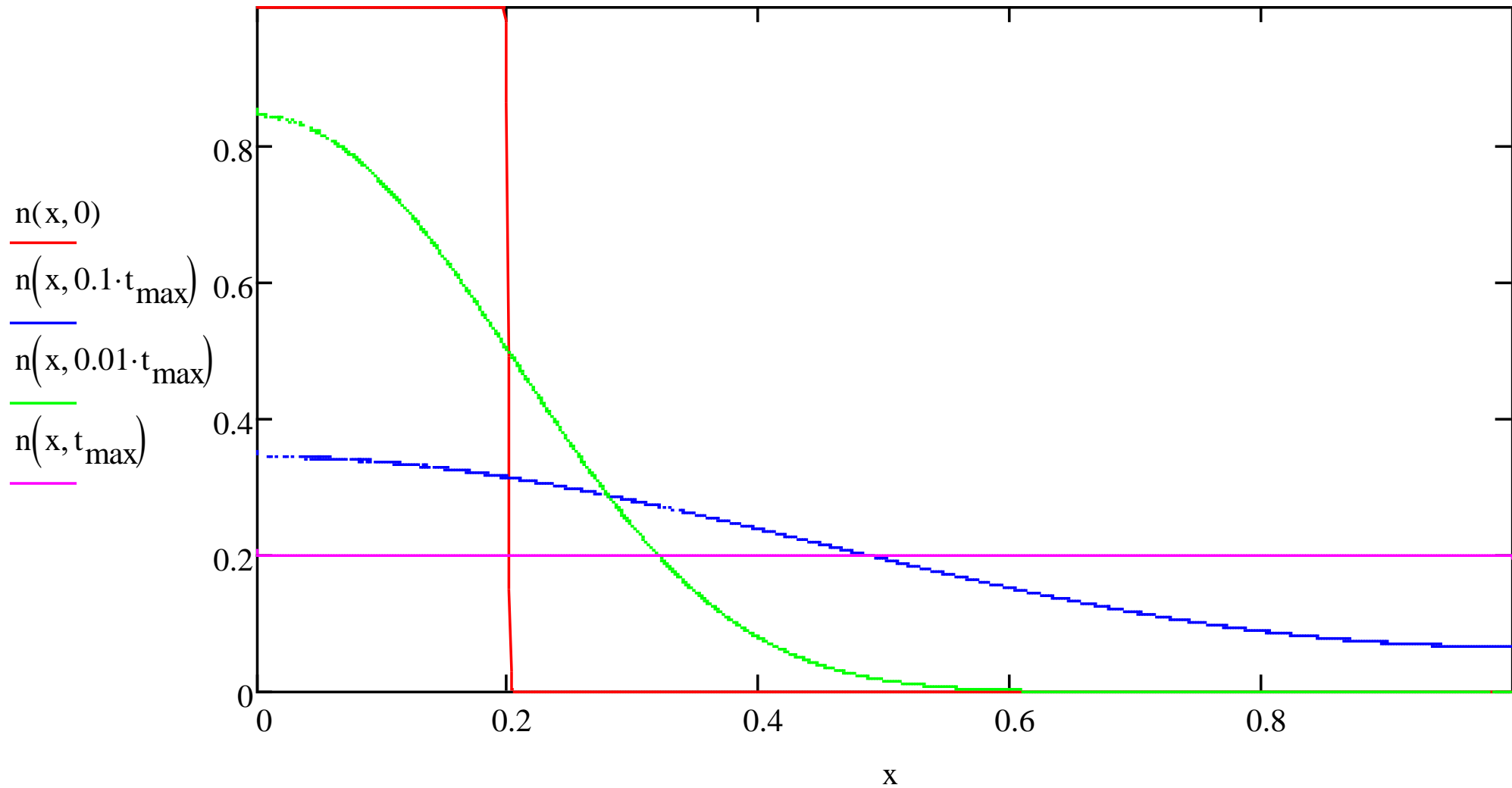
$$\frac{\partial n(\mathbf{r}, t)}{\partial t} = D \nabla^2 n(\mathbf{r}, t) - \Sigma_a \Phi(\mathbf{r}, t) + S(\mathbf{r}, t)$$

▶ Total macroscopic absorption cross section for nuclear density N : $\Sigma_a = N \sigma_a$

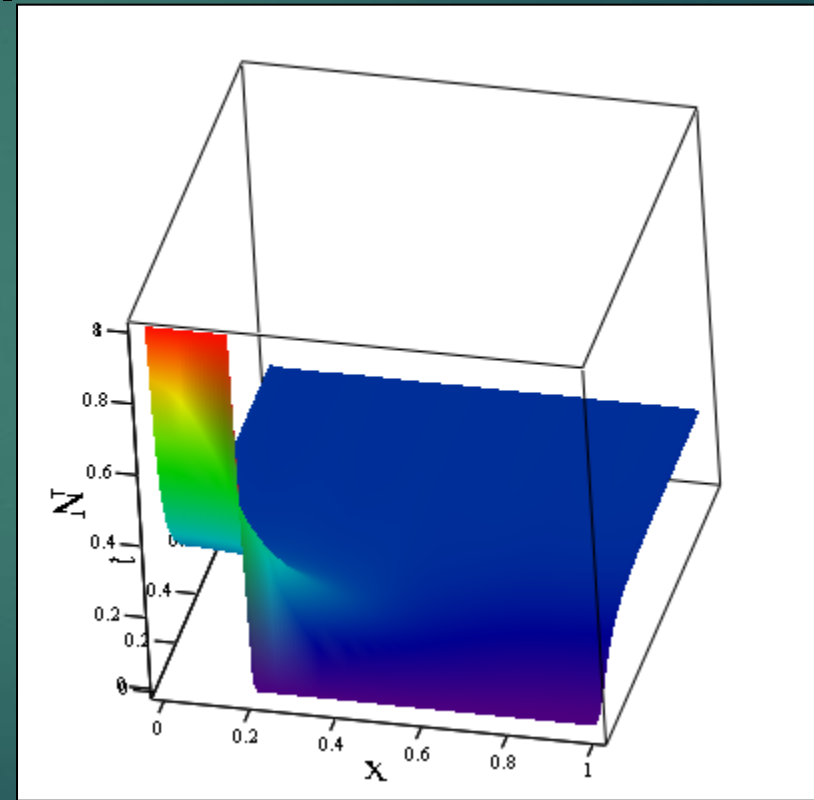
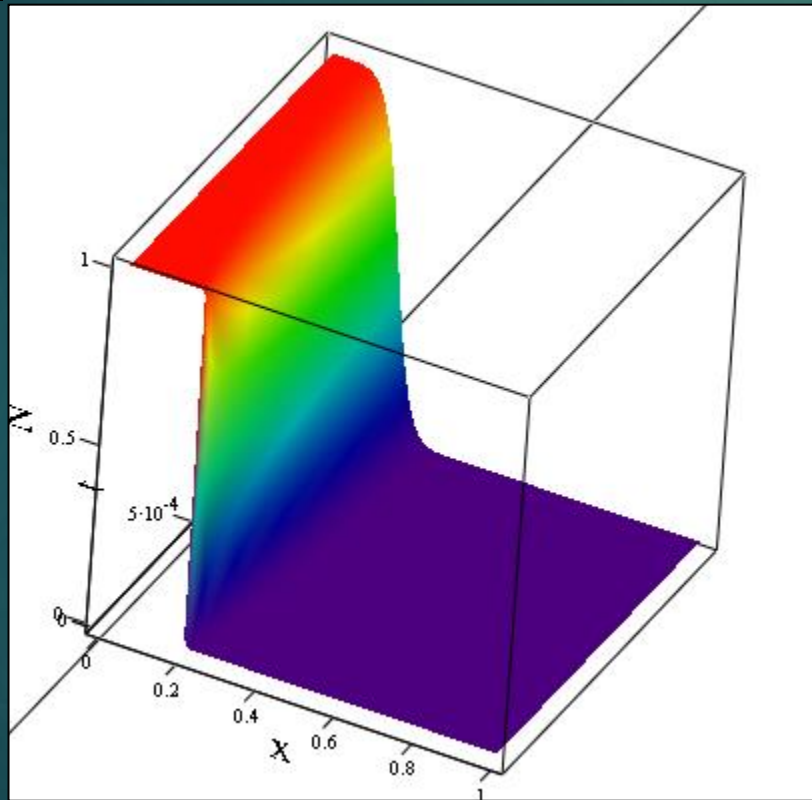
Constant or Infinite Source



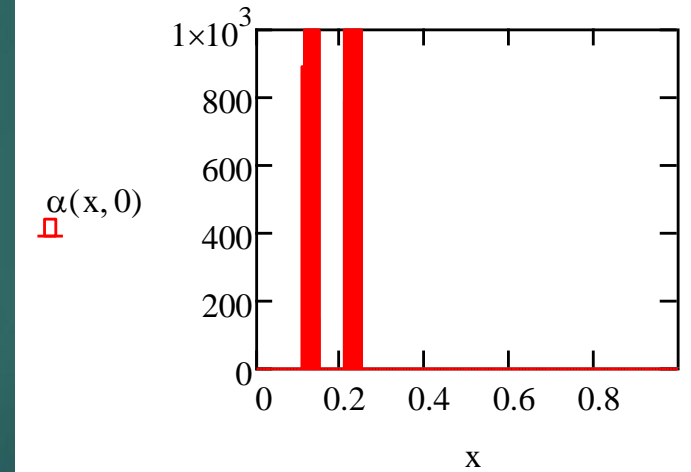
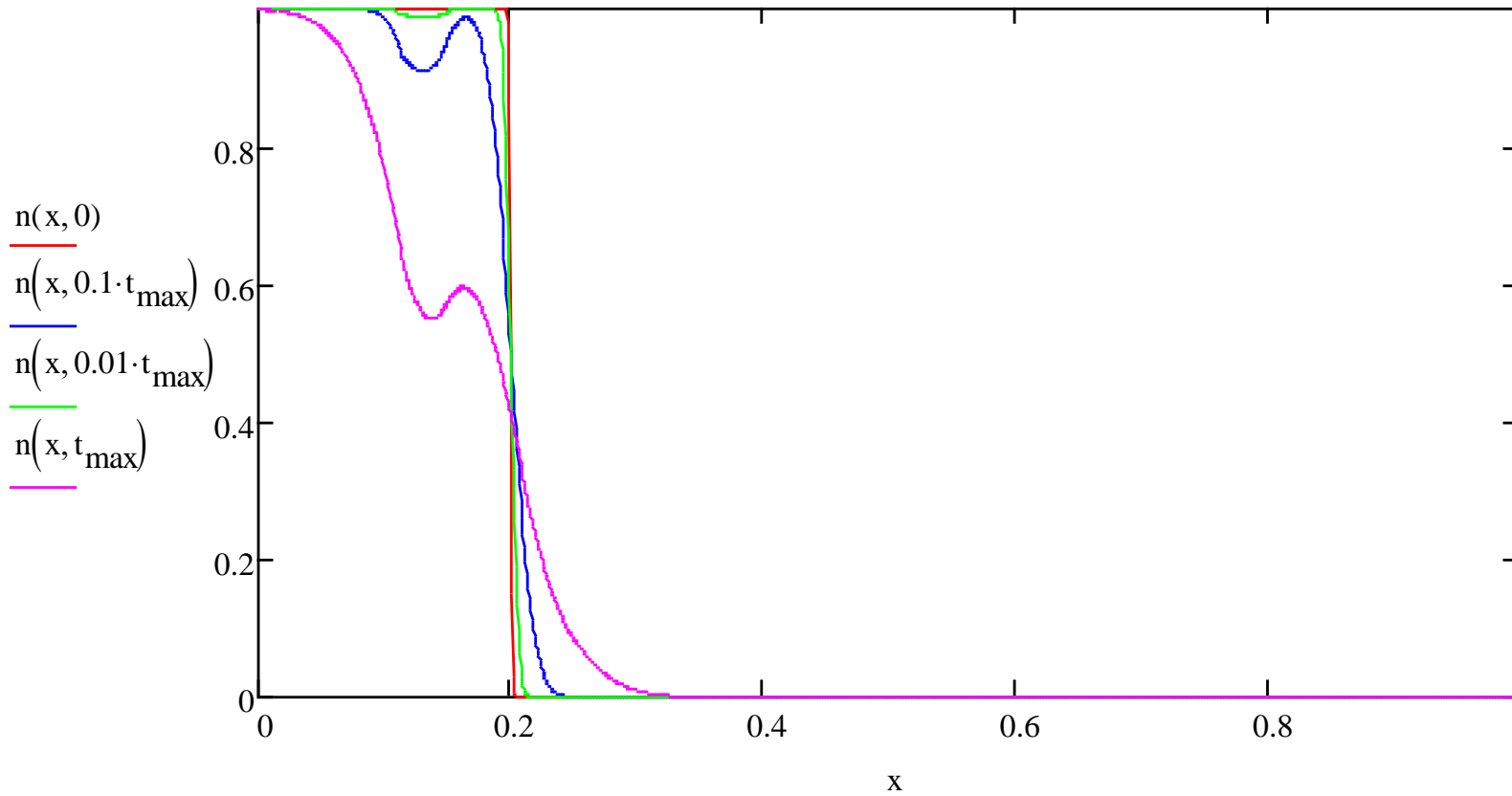
Finite Source



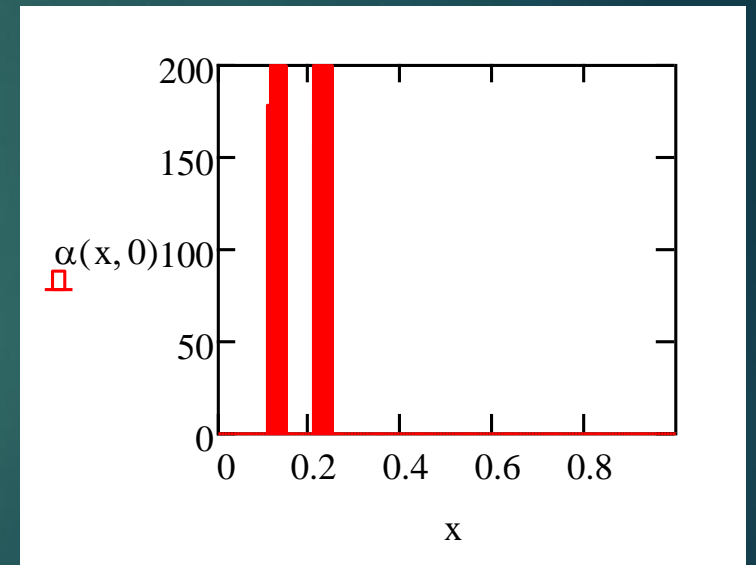
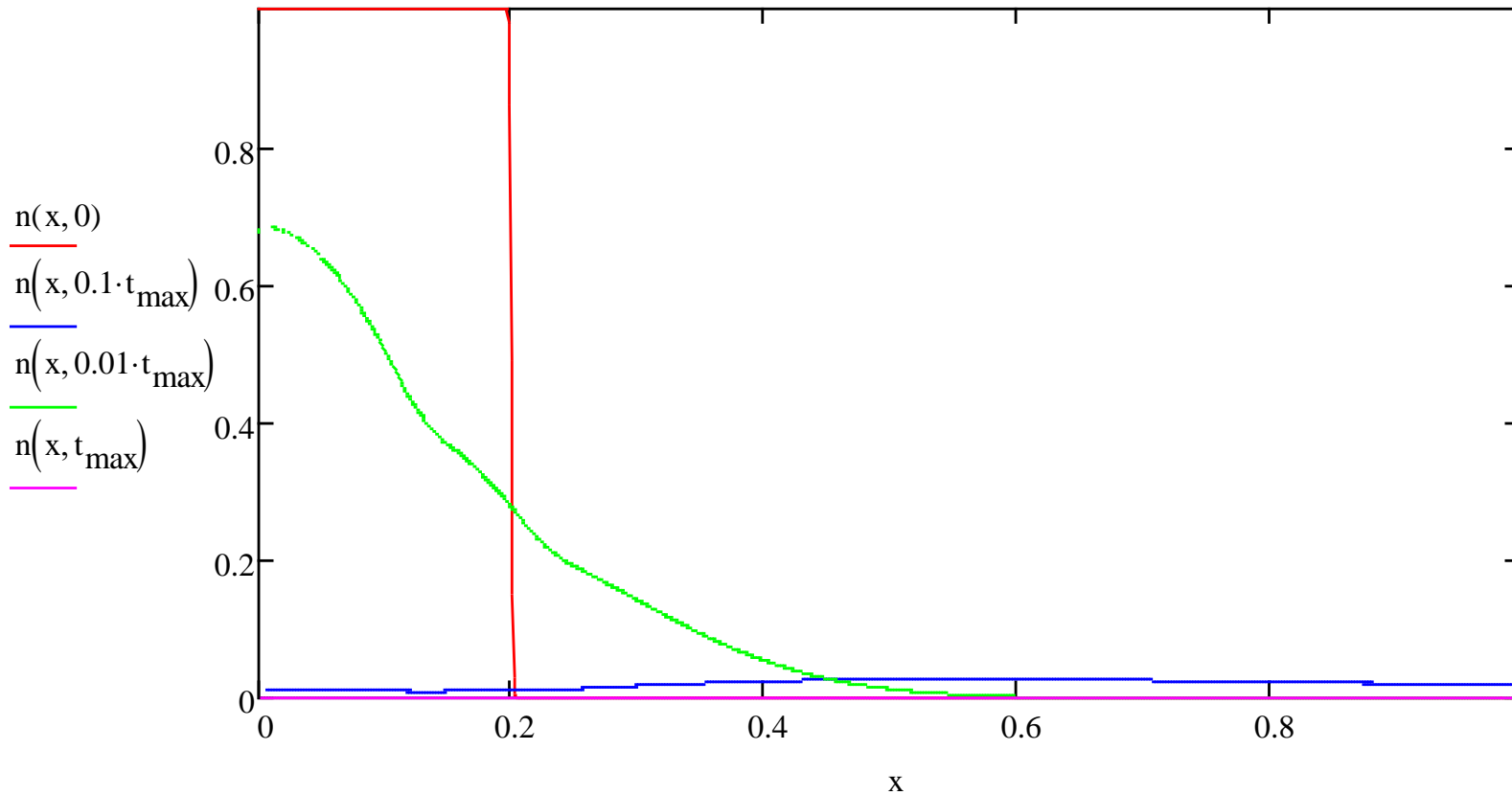
Constant and Finite Source Solutions (only diffusion) – Side by Side



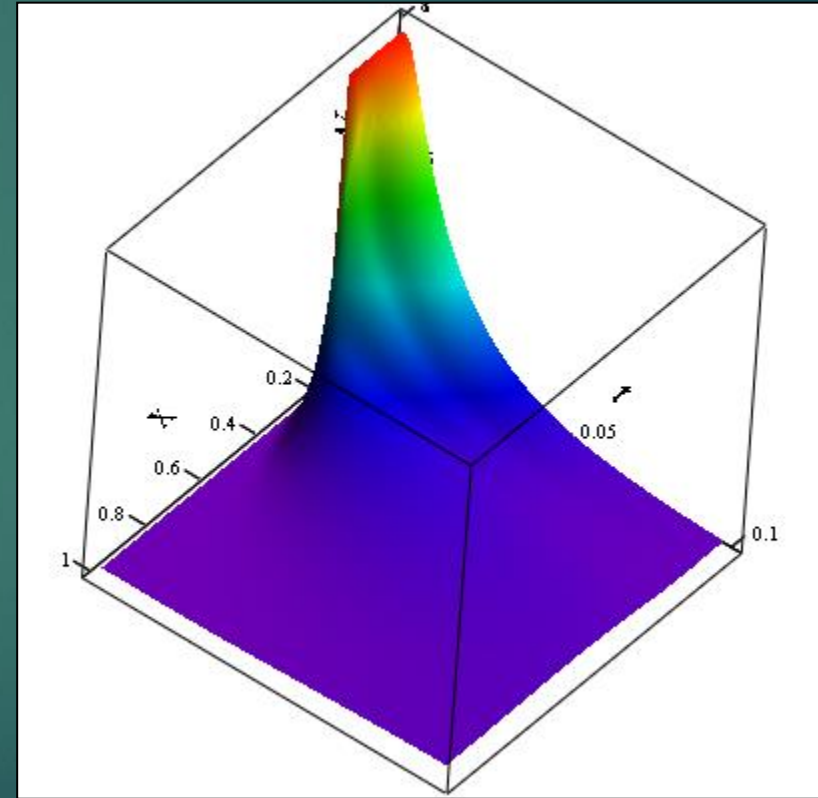
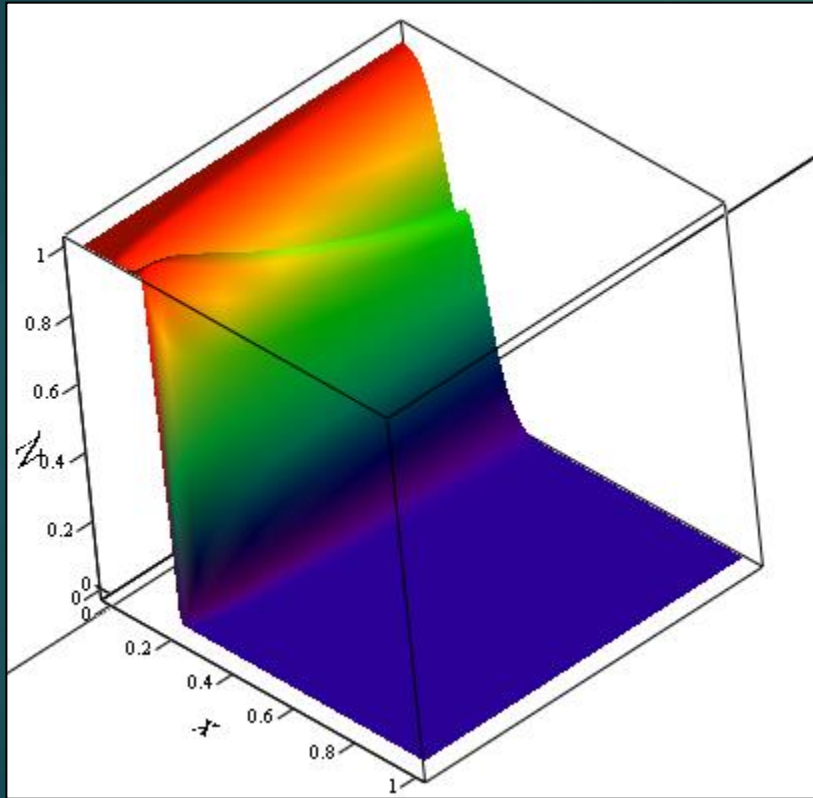
Continuity Equation with Absorption: Constant Source Case



Continuity Equation with Absorption: Finite Source Case



Constant and Finite Source Solutions (with absorption) – Side by Side



Steady-state Continuity Equation

- ▶ With no sources, other than at the boundary $\mathbf{r} = 0$, within the moderator M :

$$\Delta\Phi(\mathbf{r}) = \frac{\nu\Sigma_a(M)}{D}\Phi(\mathbf{r}) = \frac{\Phi(\mathbf{r})}{L^2}$$

- ▶ Where L is nothing but the diffusion length in the moderator M :

$$L^2 = \frac{D}{\nu\Sigma_a(M)}$$

- ▶ Another relation can be established with the neutron 'lifetime' in the moderator τ :

$$\sqrt{\frac{D}{\nu\Sigma_a(M)}} = \sqrt{D\tau} \qquad \tau = \frac{1}{\nu\Sigma_a(M)}$$

The Gradient Operator in Different Coordinate Systems

- ▶ Decartian:

$$\nabla f = \frac{\partial f}{\partial x} \mathbf{i} + \frac{\partial f}{\partial y} \mathbf{j} + \frac{\partial f}{\partial z} \mathbf{k}$$

- ▶ Cylindrical:

$$\nabla f = \frac{\partial f}{\partial \rho} \mathbf{e}_\rho + \frac{1}{\rho} \frac{\partial f}{\partial \varphi} \mathbf{e}_\varphi + \frac{\partial f}{\partial z} \mathbf{e}_z$$

- ▶ Spherical:

$$\nabla f = \frac{\partial f}{\partial r} \mathbf{e}_r + \frac{1}{r} \frac{\partial f}{\partial \theta} \mathbf{e}_\theta + \frac{1}{r \sin \theta} \frac{\partial f}{\partial \varphi} \mathbf{e}_\varphi$$

The Laplace Operator in Different Coordinate Systems

- ▶ Decartian:

$$\Delta f = \frac{\partial^2 f}{\partial x^2} + \frac{\partial^2 f}{\partial y^2} + \frac{\partial^2 f}{\partial z^2}$$

- ▶ Cylindrical:

$$\Delta f = \frac{1}{\rho} \frac{\partial}{\partial \rho} \left(\rho \frac{\partial f}{\partial \rho} \right) + \frac{1}{\rho^2} \frac{\partial^2 f}{\partial \varphi^2} + \frac{\partial^2 f}{\partial z^2}$$

- ▶ Spherical:

$$\Delta f = \frac{1}{r^2} \frac{\partial}{\partial r} \left(r^2 \frac{\partial f}{\partial r} \right) + \frac{1}{r^2 \sin \theta} \frac{\partial}{\partial \theta} \left(\sin \theta \frac{\partial f}{\partial \theta} \right) + \frac{1}{r^2 \sin^2 \theta} \left(\sin \theta \frac{\partial^2 f}{\partial \varphi^2} \right)$$

Stationery Isotropic Diffusion in Spherical Coordinates

- ▶ The Laplacian equation becomes:

$$\Delta\Phi = \frac{\partial^2\Phi}{\partial r^2} + \frac{2}{r} \frac{\partial\Phi}{\partial r} = \frac{\Phi}{L^2}$$

- ▶ Substituting in with a transformed flux:

$$u(r) = r\Phi(r)$$

$$\frac{\partial^2 u(r)}{\partial r^2} + \frac{u(r)}{L^2} = 0$$

- ▶ The general solution is:

$$u(r) = ae^{-r/L} + be^{+r/L} \quad \Phi(r) = \frac{ae^{-r/L}}{r} + \frac{be^{+r/L}}{r}$$

$$\lim_{r \rightarrow \infty} \Phi(r) = 0 \implies b = 0$$

Mean Square Distance Travelled by a Neutron in Medium

- ▶ Performing the averaging:

$$\langle r^2 \rangle = \frac{\int_0^\infty r^2 \Phi(r) dV}{\int_0^\infty \Phi(r) dV} = \frac{\int_0^\infty r^3 e^{-r/L} dr}{\int_0^\infty r e^{-r/L} dr} = 6L^2$$

- ▶ With a volume element:

$$dV = 4\pi r^2 dr$$

- ▶ The root mean square distance is therefore:

$$r_{\text{rms}} = \sqrt{6L} \cong 2.5L$$